# CURRENT TASKS OF THE DECOMMISSIONING PLANNING FOR THE KIEV'S RESEARCH REACTOR WWR-M

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## The main nuclear installations in Ukraine







WWER-440 (Rovno NPP)



WWER-1000 (Zaporizhzhya NPP, Rovno NPP, South Ukraine NPP, Khmelnitsky NPP) PEMK-1000 (Chornobyl NPP)



Research reactor (Nuclear Research Institute in Kiev, Sevastopol Nuclear Energy Institute)



State special enterprises for RW management "Radon"



Uranium mining enterprises

## OVERVIEW OF THE REGULATORY FRAMEWORK

As a whole, the normative-legal basis of Ukraine is sufficient for the decision on present-day tasks connected with the provision of safety and protection of the personnel, population and environment at the decommissioning of NPPs and RRs in Ukraine. In this area the normative-legal basis is corresponding to the international practice, accounts the recommendations of IAEA, ICRP and other international organizations.

The definition of term "decommissioning":

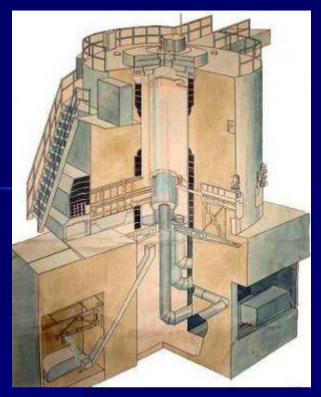
Decommissioning means such set of measures after nuclear fuel removal that excludes the operation of the facility in purposes for which it was constructed and provides personnel and the public safety and the environment security.

Following to this definition, the goal, scope and possible ways are determined:

- Decommissioning of the facility is undertaken for the excluding of the possibility of further use of the given facility with the purposes for which it was constructed.
- Decommissioning of the facility is undertaken for the achievement such site conditions that reduces maximally any restriction on the site use. It provides for:
- stage-by-stage removal of the sources of ionizing radiation being subject to regulatory control;
- abolishment of the restriction regime and reduction of radiation monitoring in the supervision zone and sanitary-protective zone of the facility.

# Normative Document NP 306.2.02/1/004-98 "General Provisions on Safety Assurance of Decommissioning of NNPs and RRs"

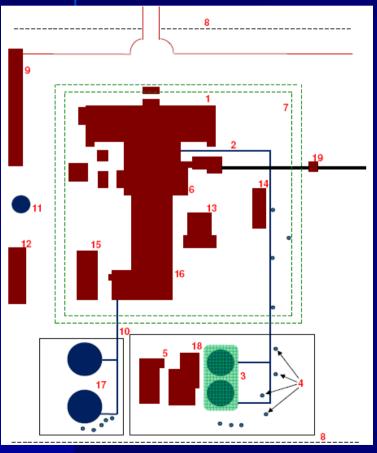
Fir	-	iF							
shut-down removal									
Operational		Decommissioning			Cancelation				
licence		licence			0	f licence			
	Permission for the termination of operation stage	Permission for the final closure stage	Permission for the dismantling stage						
Reactor is in operation	Reactor is finally shut-down					Unlimited use			
	SF in pond	SF outside reactor				of the site			
DECOMMISSIONING CONCEPT	DECOMMISSIONING PROGRAM								
	Implementation Program for the termination of operation stage	Implementation Program for the final closure stage	Implementation Program for the dismantling stage	Final report					
	SAR for the termination of operation stage	SAR for the final closure stage	<b>SAR</b> for the dismantling stage						
	Technological rules for the termination of operation stage	Technological rules for the final closure stage	Technological rules for the dismantling stage						



## **Current status of reactor**

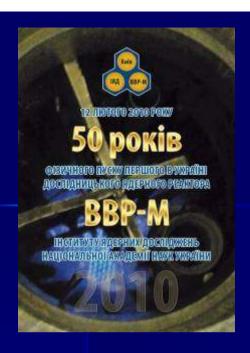
The WWR-M reactor is a heterogeneous water-moderated pool type research reactor operating with thermal neutrons at a power of 10 MW<sub>th</sub>, giving a maximum neutron flux of 1.5×10<sup>14</sup> cm<sup>-2</sup>s<sup>-1</sup> at the core center.





# Layout of the buildings and constructions on the reactor site:

- 1 reactor building;
- 2 reactor sewage system pipeline;
- 3 reactor waste tanks;
- 4 observation holes;
- 5 workshop;
- 6 tambour of reactor hall;
- 7 physical protection fence;
- 8 site wall:
- 9 store building;
- 10 hot cells sewage system pipeline;
- 11 water-tower:
- 12 cooling tower;
- 13 ventilation center;
- 14 gasholder;
- 15 secondary circuit pump-house;
- 16 hot cells building;
- 17 hot cells waste tanks;
- 18 evaporator installation;
- 19 experimental hall on the horizontal channel



"Strategic Plan for the use of research reactor WWR-M of the Institute for Nuclear Research" was approved by the National Academy of Sciences of Ukraine in July 2004.

Plan determines the strategic goal as the <u>extension of the reactor</u> <u>operation till 2015</u>



The term of operation was continued by the decree of the SNRCU's Board (*No 11 from 21 May 2009*) and then a new permission for the reactor operation will be issued:

- to continue the reactor operation till 31.12.2013;
- to convert the reactor on the stage "termination of operation" from 1.01.2014;
- in the case of the operator's decision concerning the reactor's further operation, to prepare and agreed with the SNCRU the possibilities and conditions of the reactor operation.

## CURRENT STATUS OF DECOMMISSIONING PLANNING

Preparation for the further decommissioning of the Kiev's research reactor was started in the framework of the **Decommissioning Concept** issued in 2001.

This document contains a common decommissioning approach and measures, which must be detailed and updated with the goal of preparation of the justified decommissioning plan.

The next step of decommissioning planning is the development of the detailed *Decommissioning Program*, which was elaborated during 2007-2009 and approved by the Regulatory Body decision from 4 November 2009.

This document determines and substantiates the main technical and organizational measures for the decommissioning preparation and implementation, the sequence of works and measures, necessary conditions for their execution and provision. Special attention is concentrated on the *ecological safety* of decommissioning due to the reactor location in the megapolis.

The most likely option will be the use of the reactor's building with the "hot cells" as the separate laboratory for the development and application of radiation technologies.

The variant of *immediate dismantling* was selected for the WWR-M reasoning based on the plans for the further site use with the removal of spent fuel and radwaste outside Kiev and site return for the unrestricted use.

In accordance with the preliminary estimations, the decommissioning timeframe will not exceed 6 years.

## **DECOMMISSIONING PROGRAM of the RESEARCH REACTOR WWR-M (2009)**

#### 1. MAIN CONDITIONS OF PROGRAM

- 1.1. Introduction
- 1.2. Purpose, goal and tasks of the Program
- 1.3. Basic regulations
- 1.4. Scope
- 1.5. Revision of Program

#### 2. DESCRIPTION OF THE REACTOR SITE

- 2.1. Geographical and demographical characteristics of the environment
- 2,2, Meteorology
- 2.3. Hydrology
- 2.4. Geology and seismology

#### 3. REACTOR DESCRIPTION

- 3.1. Purpose, type and technical parameters of the reactor
- 3.2. Reactor design
- 3.3. Main buildings
- 3.4.Auxiliary buildings
- 3.5. Building construction state and conditions on the reactor site

#### 4. HISTORY OF REACTOR OPERATION

- 4.1. Reactor utilization
- 4.2. Reactor operation
- 4.3. Modernization of the reactor systems and refurbishment of the equipment
- 4.4. Reactor accidents and personnel dose loading
- 4.5. Records on reactor inspections
- 4.6. Radiation conditions at the reactor
  - 4.6.1. Radiation conditions in the reactor hall
  - 4.6.2. Radiation conditions in the reactor building
  - 4.6.3. Radiation conditions in the pump house and adjacent premises
  - 4.6.4. Radiation conditions around reactor

#### 5. PLANS FOR USE OF THE SITE, MATERIALS AND COMPONENTS

- 5.1. Plans for use of the site
- 5.2. Plans for use of the materials and components

#### 6. DECOMMISSIONING STRATEGY

#### 7. SEQUENCE OF DECOMMISSIONING

- 7.1. Activity for the preparation of decommissioning at the reactor operation
- 7.2. Termination of operation
- 7.3. Final closure
- 7.4. Dismantling

#### 8. RADWASTE MANAGEMENT PROGRAM

- 8.1. Solid RAW generation and accumulation
- 8.2. Liquid RAW generation and accumulation
- 8.3. Order of the RAW transfer to the special enterprise
- 8.4. Organizational scheme for the RAW management
- 8.5. RAW account and control
- 8.6. Radiation control at the RAW management
- 8.7. Analysis of accidents at the RAW management
- 8.8. Quality assurance at the RAW management
- 8.9. Measures for the RAW minimization
- 8,10. Implementation of the modern methods for the RAW management
- 8.11. Decommissioning radwaste

#### 9. SAFETY PROVISION

- 9.1. Nuclear safety
- 9.2. Radiation safety
- 9.3. Physical protection system
- 9.4. Fire safety
- 9.5. Industrial safety
- 9.6. External monitoring
- 9.7. Emergency response

#### 10. PROGRAM OF RADIATION PROTECTION

- 10.1. Control irradiation levels for the personnel
- 10.2. System of radiation and dosimeter control
- 10.3. Control of the staff external exposure
- 10.4. Control of the radioactive effluents
- 10.5. Organizational measures aimed on the irradiation dose reduction 10.6. Modernization of the radiation control system
- 10.7. Radiation protection system at the reactor decommissioning

#### 11. SUPPORT INFRASTRUCTURE FOR THE DECOMMISSIONING WORKS

- 11.1. Power supply
- 11.2. Water supply
- 11.3. Heat supply
- 11.4. Sewerage system
- 11.5. Mechanical workshop
- 11.6. Roads and transportation

#### 12. DECOMMISSIONING PLANNING AND MANAGEMENT

- 12.1. Technical management 12.2. Scientific support
- 12.3. Administration
- 12,3, Administration
- 12,4. Quality assurance group

#### 13. DOCUMENTATION FOR THE DECOMMISSIONIG

- 14. QUALITY ASSURANCE
- 15. PERSONNEL UTILIZATION
- 16. SOCIAL PROTECTION OF THE REACTOR STAFF
- 17. PUBLIC RELATIONS ON THE DECOMMISSIONING PROBLEMS

#### ATTACHEMENTS:

- 1 Legislation and normative documents
- 2 The results of environmental external radiation monitoring in the WWR-M reactor's zone of influence
- 3 "Hot cells" application
- 4 Peculiarities of EIA at the WWR-M reactor decommissioning
- 5 Preliminary program of the complex engineering and radiation inspection
- 6 List of the reactor system's equipment
- 7 List of the radiation control system equipment,

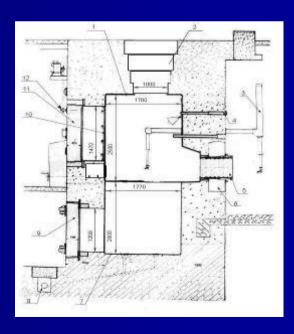
## **Radiation Material Study Department**

## **Available facilities:**

- heavy hot cells allowing the investigations of samples with the activity up to 25000 Ci;
- ight hot cells (up to 250 Ci).

The laboratories are fitted out with the remotely-operated equipment for the study of physical-mechanical characteristics of materials irradiated with the high neutron doses, surveillance specimens of the reactor vessel metals of the Ukrainian NPPs.





## Radiation Material Study Department

INR has been appointed as the leading organization for the scientific support of the safe operation of the WWER-440 and WWER-1000 type reactor vessels.

Main directions of scientific activity:

- Determination of mechanisms of radiation damage of the solids, selection of the most advanced structural materials for nuclear reactor industry.
- Radiation material science aspects of operating reactors safety, investigation of the radiation embrittlement of the reactor vessel steels and determination of the WWER-1000 type reactor vessels safe operation lifetime in particular.

#### PRINCIPAL REACTOR SYSTEMS:

	SYSTEM	Complete dismantling	Utilization at the decommissioning	Utilization after decommissioning <sup>a)</sup>
1	Reactor vessel	+	-	-
2	Primary and secondary circuits	+	-	-
3	Water cleaning system (primary circuit)	+	-	-
4	Control rod system	+	-	-
5	Water supply system	+	-	-
6	Emergency cooling system	+	-	-
7	Electric power supply system	-	+	?
8	Special ventilation system	·	+	+
9	Radiation protection system	+	-	-
10	Radiation control system система	-	+	+
11	Special sewerage system	-	+	+
12	Fresh fuel storage	+	-	-
13	Spent fuel storage (CP-1)	+	-	-
14	Spent fuel storage (CP-2)	-	-	?
15	Experimental installations	+	-	-
16	"Hot-cells" (4 cells)	+	-	-
17	Fire protection system	-	+	+
18	Physical protection system	-	+	+
19	Loading tools (cranes)	-	+	+
20	Heat supply system	-	+	+

<sup>&</sup>lt;sup>a)</sup> in accordance with accepted decision for the utilization of the reactor building with the "hot-cells" as the separate laboratory (after the transfer of the main reactor building, the part of existing infrastructure and the auxiliary buildings to such laboratory)

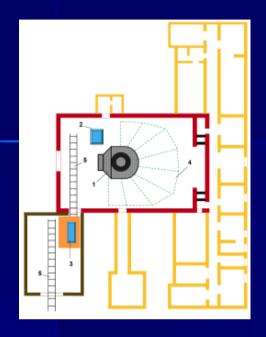
# Activity for the preparation of decommissioning during the reactor operation

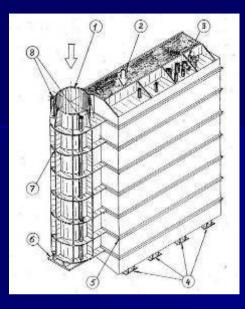
- At the reactor operation it will be performed the set of works directed toward the preparation of decommissioning. The following measures are carried out permanently:
- classification, accounting and forecast of the radwaste volumes, which will be generated during the reactor operation and decommissioning;
- collection, processing and storage of information related to the buildings, the constructions and the reactor systems and elements, which will be required for the reactor decommissioning;
- works aimed on the preparation and removal of the spent nuclear fuel;
- gathering of the material and technical resources for the decommissioning;
- development of the decommissioning documentation;
- request and approval of the decommissioning license;
- public relations on the decommissioning problems.

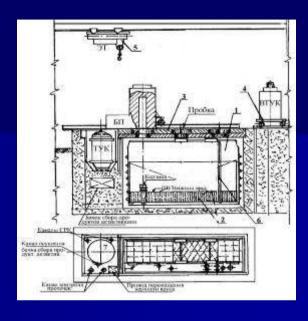
Main material	Weight (kg)						
REACTOR WWR-M							
CAB-1	3600						
beryllium	835						
steel, cast iron, graphite	15000						
steel	3884						
steel, cast iron, lead	65000						
steel, cast iron	30550						
CAB-1, cast iron	4220						
steel 1X18H9T	1880×4=7520						
CAB-1, steel 1X18H9T	19000						
Total weight of materials (kg): 149609							
PRIMARY CIRCUIT							
steel 12X18H10T	6044						
steel 1X18H9T	837						
steel 1X18H9T	5320						
steel 12X18H10T	2×7694=15388						
steel 12X18H10T	1800						
steel 12X18H10T	1337						
steel 12X18H10T	1700						
Total weight of materials (kg): 32426							
SECONDARY CIRCUIT							
steel 207	39222						
steel, cast iron	773×3=2320						
cast iron	7659						
Total weight of materials (kg): 49201							
AUXILIARY SYSTEMS							
steel 1X18H9T	12748						
steel 1X18H9T	22576						
brick	23580						
steel 1X18H9T	19450						
steel 3	3200						
Total weight of materials (kg): 81554							
	CAB-1 beryllium steel, cast iron, graphite steel steel, cast iron, lead steel, cast iron CAB-1, cast iron steel 1X18H9T CAB-1, steel 1X18H9T  Total weight of materia  RY CIRCUIT steel 12X18H10T steel 1X18H9T steel 12X18H10T						

TOTAL (kg): 312790

## **WORKS CARRIED OUT DURING 2009 - new SFSF**















## SF reloading operations (November 2009)













## SF shipment (May 2010)







#### **AVAILABLE DECOMMISSIONING EXPERIENCE**

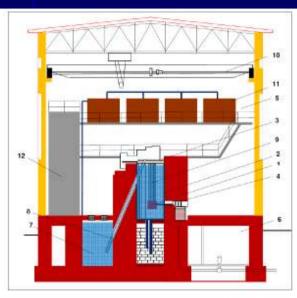


Fig.6. Vertical cross-section of the reactor building (axes: spent fuel storage tank – center of core – reactor hall – primary circuit pump house). Legend: 1 – reactor core; 2 – reactor vessel; 3 – reactor cover plate; 4 – gate of horizontal experimental channel; 5 – reactor hall (height of 16.1 m); 6 – pump house of primary circuit; 7 – cooling pond; 8 – transport channel for spent fuel; 9 – control rod channels and vertical experimental channels; 10 – bridge crane (lifting power of 10 t); 11 – tanks with the reserve of distilled water; 12 – scrubber.



The exchange of heat-exchangers (4×95=380  $\text{M}^2 \rightarrow 2\times329=658 \text{ M}^2$ ) and pipe-line segments.

#### Sequence:

- the reactor core reloading and removal of heatcarrier to the special drainage system;
- the dismantling of experimental equipment at the horizontal channels located above the primary circuit pump-house;
- the dismantling of covering plates in the primary circuit pump-house;
- the dismantling of equipment and pipelines, which are a subject of replacement in the primary circuit pump-house;
- the removal of dismantled equipment from the primary circuit pump-house and reactor hall by means of load-lifting mechanisms (crane 10 t; telpher 10 t);
- the decontamination of external surfaces of the dismantled equipment and pipelines; packing into protective film;
- the loading on special transport car in the reactor hall tambour and transportation to the disposal place.

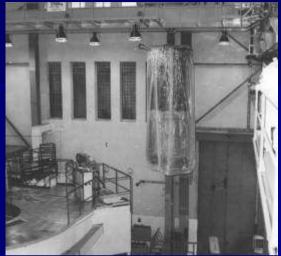
Total amount of removed contaminated equipment:

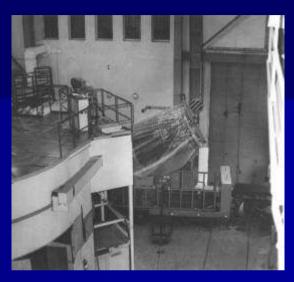
- heat-exchanger 4 items 5000 kg×4 = 20000 kg
- pipelines Ø 325×12 = 3500 kg

Altogether: 23500 kg

## Removal of the reactor vessel (Rez, 1989; Rossendorf 2002)













## FURTHER PLANS AND EXPECTED DIFFICULTIES

- removal of HEU fuel (completed in May 2010);
- elaboration and partial implementation of the CERI Program (Complex engineering and radiation inspection = Radiological survey);
- sequence of dismantling procedures (inverted sequence of assembling); dismantling design:
  - reactor internals;
  - reactor vessel;
  - concrete biological shield;
  - pump-house and primary circuit;
- SAR and EIA for decommissioning stages;
- cost estimations for sub-options

#### STRUCTURE OF DECOMMISSIONING DOCUMENTATION ASSEMBLING DECOMMISSIONING OPERATIOANAL DRAWINGS PROGRAM TECHNOLOGICAL (Draft of dismantling) (1976) (2009)RULES (2009) PROGRAM of CERI 2010 CERI REPORT SF storage (CP-2) PROJECT of SEPARATE DISMANTLING LABORATORY DESIGN DECOMMISSIONING TECHNOLOGICAL RULES FINAL DECOMMISSIONING SAR for decommissioning PROGRAM EIA for decommissioning Termination of Final closure Dismantling operation stage stage stage 2012 2013

## **Summary**

- Further reactor operation is assumed now as the main task.
   Operation license will be in force till 2014.
- Decommissioning Program was approved by Regulatory Body at the end of 2009. This DP is considered as the intermediate stage of decommissioning planning.
- Development of necessary decommissioning documents and detail time schedule is in progress now. Prioritisation for such development is established.

### **AVAILABLE PUBLICATIONS**

- 1) Yu.N.Lobach, E.V. Svarichevskaya, V.V.Trishin

  Peculiarities of the environmental impact assessment at the decommissioning of the research reactor WWR-M

  Nuclear and radiation safety, 11, No3 (2008) p.29-34
- 2) Yu.N.Lobach, T.G.Ludanova, M.V.Lysenko, V.N.Makarovsky, V.N.Shevel

  Preparation for Decommissioning of WWR-M

  Proceedings of the Int. Conf. "Decommissioning Challenges: an Industrial Reality?" (1)

**Proceedings** of the Int. Conf. "Decommissioning Challenges: an Industrial Reality?", Sept. 28 to Oct.2, 2008, Avignon, ref.008

3) Yu.N.Lobach

Decommissioning planning for the Kiev's research reactor WWR-M, in IAEA-TECDOC-1602 Innovative and adaptive technologies in decommissioning of nuclear facilities, 2008, p.251-266

- 4) Yu.N.Lobach, V.N.Makarovsky, V.N.Shevel

  Radwaste management at the decommissioning of the WWR-M reactor

  Proceedings of the 10th Int.Conference "Problems of the radioactive waste management in Ukraine", 1-3 Oct. 2008,

  Kyiv, p.21-25
- 5) Yu.N.Lobach, T.G.Ludanova, M.V.Lysenko, V.N.Makarovsky, V.N.Shevel *Principal provisions of the decommissioning program for the WWR-M reactor* **Proceedings** of the 2nd Int. Conf."Current Problems in Nuclear Physics and Atomic Energy", 9-15 June, 2008, Kyiv, p.670-673
- 6) Yu.N.Lobach, V.N.Shevel
  Radiation protection tasks on the Kiev's research reactor WWR-M
  Nuclear technology and radiation protection, 24, No2 (2009) p.145-151
- 7) Yu.N.Lobach, M.V.Lysenko, V.N.Makarovsky
  Substantiation of the decommissioning strategy selection for the research nuclear reactor WWR-M
  Nuclear and radiation safety, 12, No3 (2009) p.46-51

