International Conference on Control and Management of Inadvertent Radioactive Material in Scrap Metal Spanish Nuclear Safety Council; Tarragona, Spain; 23-27 Feb. 2009

Exclusion, Exemption and Clearance in the frame of the control and management of inadvertent radioactive material in scrap metal (Exclusión, exención y desclasificación en el marco del control y la gestión de material radiactivo inadvertidamente presente en la chatarra)

Abel J. González

Representative to the United Nations Scientific Committee on the Effects of Atomic Radiation (UNSCEAR) Vice-President of the International Commission on Radiological Protection (ICRP) Member of the Commission of Safety Standards of the IAEA

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International Conference on **Control and Management of Inadvertent Radioactive Material in Scrap Metal**

THE PARTY OF

Tarragona, Spain 23-27 February 2009

Organized by the



In cooperation with the

(IAEA ara

Co-sponsored by the

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NEA / OCDE NEA

Ministerio de Industria, Turismo y Com

82 FER Federación Española de la Recuperación

WUNESID Unión de Empresas Siderúrgicas

Empresa Nacional de Residuos Radiactivos S.A.

Diputació de Tarragona NUMBER OF T

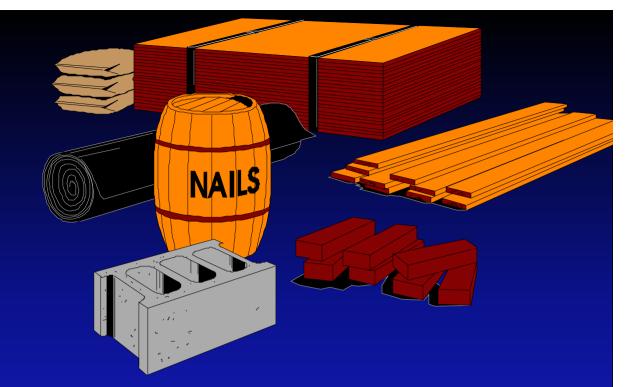
.... Ajuntament de Tarragona

C Durantera En ante Vancas Universitat Rovira i Virgili





What is the problem?



...radioactive

substances are

incorporated into

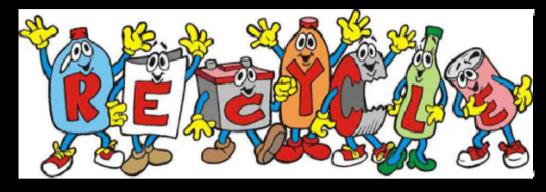
materials, goods,

merchandises,

products...

....and in scrap.....

...and, as scrap would



into new produce,

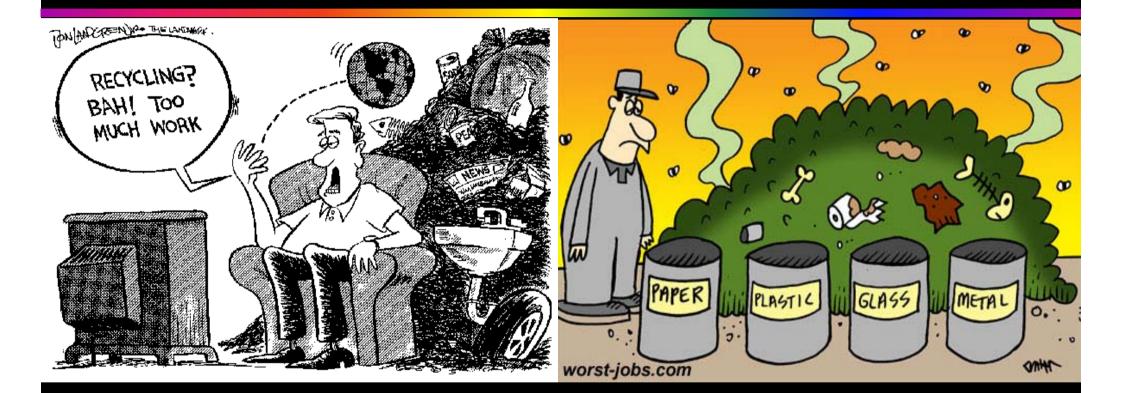
the process of introduction

of radioactive susbtanaces

into commodities will

continue

(even if you do not believe in recycling!)



Theses

 The situation is unstoppable and requires straightforward solutions.

 The problem is global and therefore the solution (s) should be global.

 A clear intergovernmental agreement is needed establishing to what extent commodities should be regulated.

Not surprisingly,

people (and their representatives)

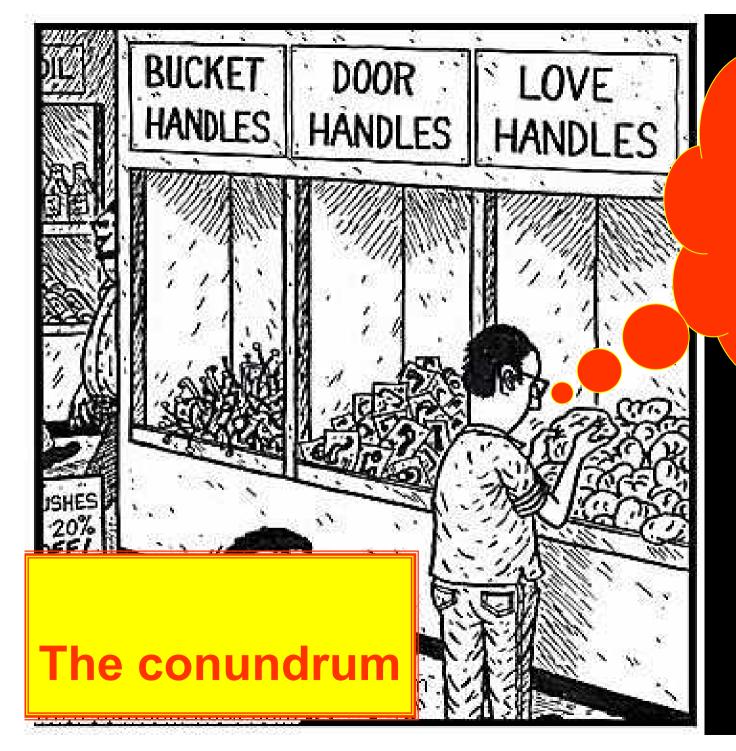
have been asking a simple basic question to the

radiation protection community:

What is the amount of radioactivity in commodities

above which radiation protection control is required?





Should this stuff be controlled by CSN?

The challenge for the RP community

 to provide a rational, logical and sustainable answer to this simple question:
 What is the safe level of radioactivity in products of public use



Otherwise people will

be convinced that

scrap metal = death

(An apparently obvious reflection)

Scrap metal (and recycled produce) may contain becquerels (or curies) but NOT sieverts (or rems)

Therefore, criteria should be set in terms of activity rather than in terms of hypothetical doses!

(Another apparently obvious reflection)

Scrap metal is an international commodity

Therefore, criteria should be internationally agreed!

HISTORY

(... at the beginning there was light!)



Roman Law (B.C.)



• DEMINIMISNON CVRATLEX

(<u>Cause</u> of no-concern for the <u>law;</u>

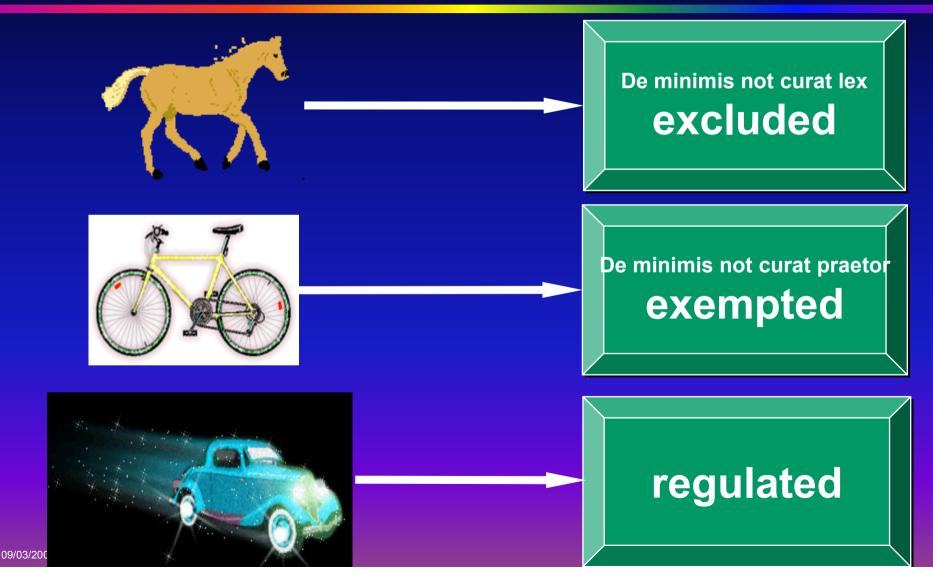
excluded from the law)

 DEMINIMISNON CVRAT PRÆTOR
 (Cause of no-concern for the regulator; exempted by the regulator)

These simple basic principles are at the basis of civil organization

(except for radiation protection)

Regulation of transportation in my village in the Argentine pampas



These simple concepts were surprisingly absent from radiation protection criteria. In 1967, there was an

attempt to introduce them.

Basic Safety Standards for Radiation Protection 1967 Edition

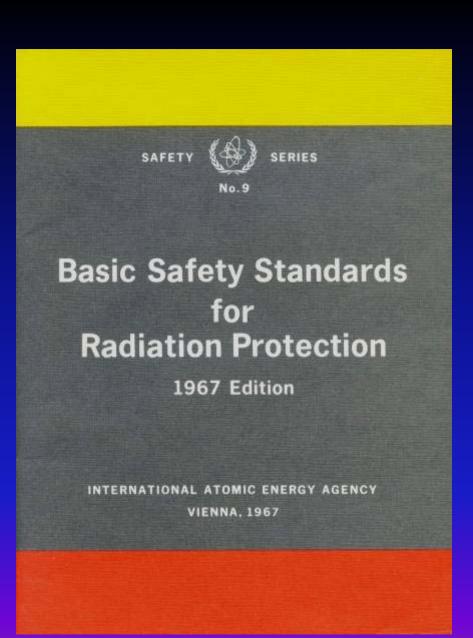
No. 3

SERIES

SAFETY

INTERNATIONAL ATOMIC ENERGY AGENCY VIENNA: 1967

The 1967 BSS '...apply to operations which do not involve the use of radioactive substances at concentrations exceeding 0.002 µCi/g (74 Bq/g) or solid natural radioactive substances at concentrations exceeding 0.01 μCi/g (370 Bq/g).'



I.e., the 1967 BSS de facto excluded operations involving

radioactive substances at concentrations below

several 10's of Bq/g!

... then, the 80's

shown a reverse in

policy: ...

... the era of purism...



SAFETY STANDARDS

Basic Safety Standards for Radiation Protection

1982 Edition

HARA, ILO, NEALOPCOL, WHO



(5) INTERNATIONAL ATOMIC ENERGY AGENCY, VIENNA, 1982

... the 1982 BSS required that everything be regulated...

... at the time, a serious

grammatical mistake

was made by lawyers...



de minimis non curat lex

and

de minimis non curat prætor,

which are both *ablative* grammatical cases,

were converted into

'<u>de minimis</u> <u>dose'</u>

i.e., the adjective de minimis qualifying the substantive dose

This mistake created a tremendous confusion...

...which still permeates into our discussions today

The 'de-minimis dose' confusion

- 1. Equates not curat lex and not curat prætor (i.e., equates legislator and regulator).
- 2. Equates cause and qualification.
- 3. Contradicts basic radiation protection principle: ...all doses...however small....(etc).
- 4. Moves focus from activity to doses

non curat law and not curat prætor

Totally different legal concepts:

Non curat lex

Control is unfeasible or unamenable (for this reason it may be excluded from the law)

Non curat prætor

Some control may be unwarranted or unnecessary (for this reason it may be exempted by the regulator)

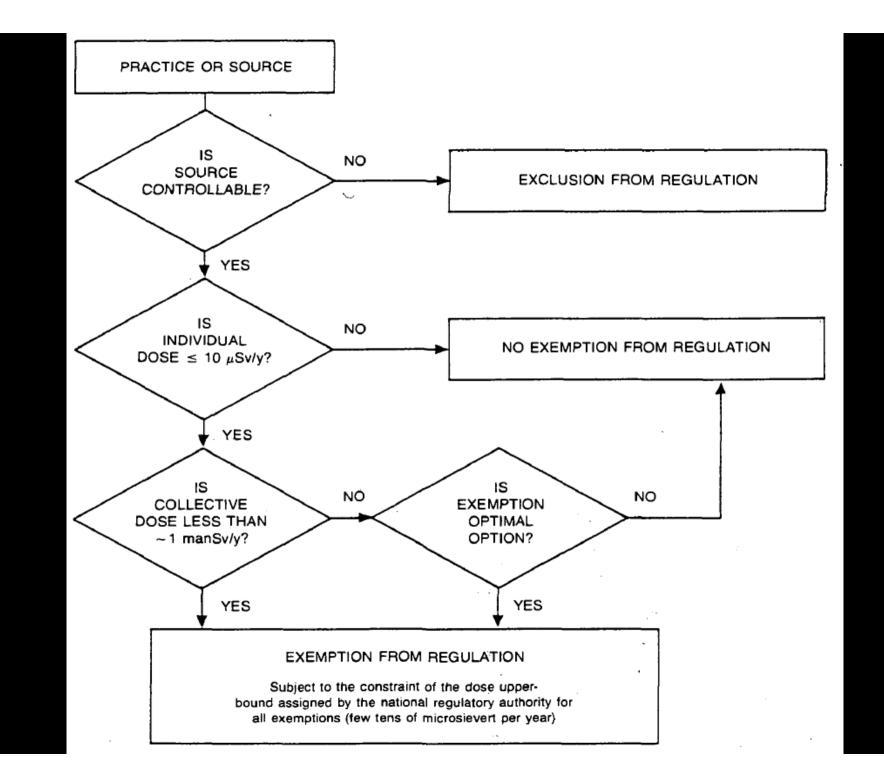
Exemption from regulatory control: An international consensus

A summary of essential features and concepts

by G.S. Linsley and A.J. González

IAEA BULLETIN, 3/1988

http://www.iaea.org/Publications/Magazines/Bulletin/Bull303/30302342730.pdf



09/03/2009

safety series

IAEA SAFETY GUIDES

Principles for the Exemption of Radiation Sources and Practices from Regulatory Control

MARA AND DECEMBER

25 September 1988

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INATIONAL ATOMIC ENERGY AGENCY, VIENNA 1988



<page-header><section-header><text> series

SAFETY SERIES No. 115

Basic Safety Standards for Protection against

JOINTLY SPONSORED BY FAO, IAEA, ILO, OECD/NEA, PAHO, WHO

INTERNATIONAL ATOMIC ENERGY AGENCY, VIENNA, 1996 IAEA-TECDOC-855

Clearance levels for radionuclides in solid materials

Application of exemption principles

Interim report for comment



INTERNATIONAL ATOMIC ENERGY AGENCY

5 February 1996

IAEA-TECDOC-1068

Application of radiological exclusion and exemption principles to sea disposal

The concept of 'de minimis' for radioactive substances under the London Convention 1972



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INTERNATIONAL ATOMIC ENERGY AGENCY

March 1999

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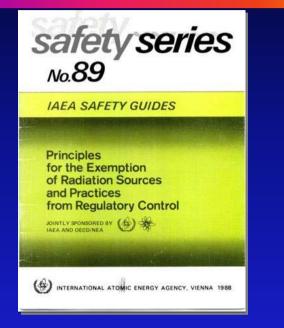
5 March 1999

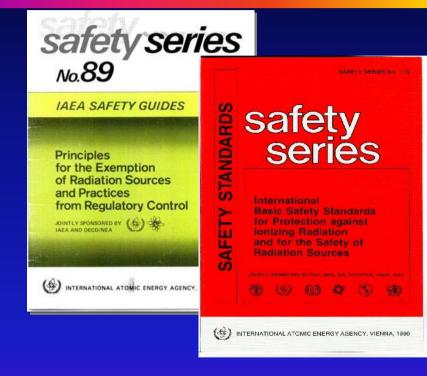
Safety Reports Series No.44

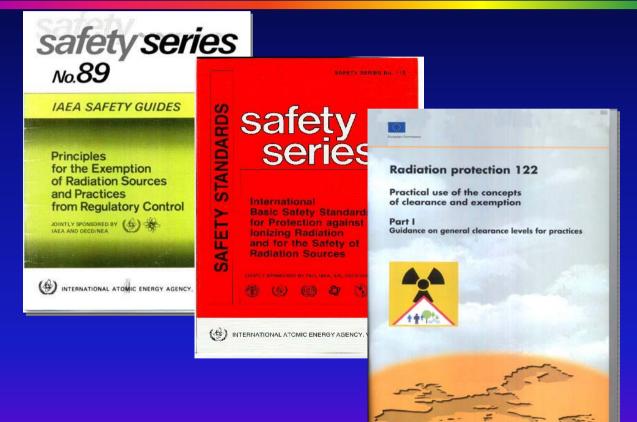
> Derivation of Activity Concentration Values for Exclusion, Exemption and Clearance

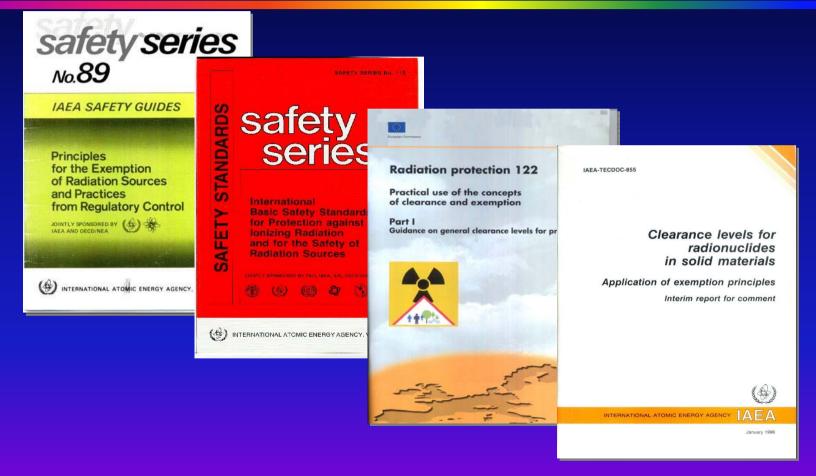


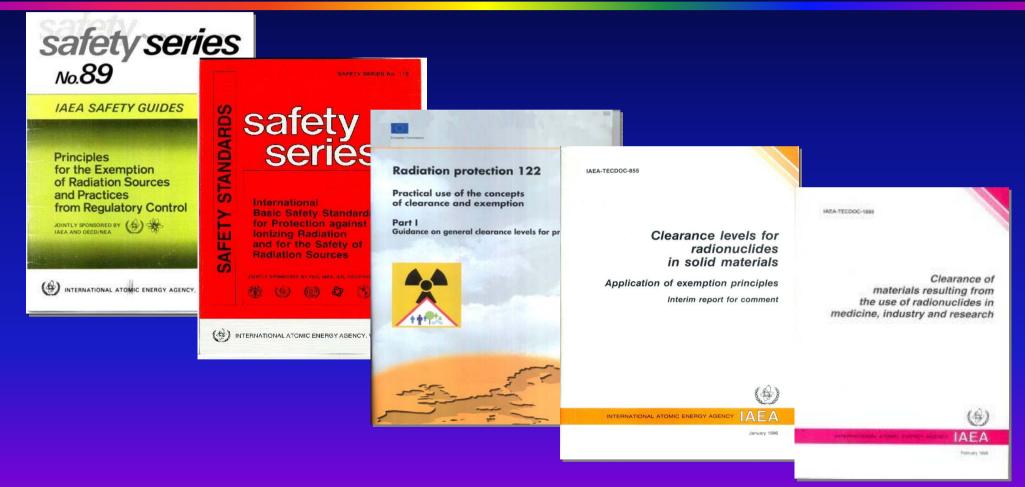
25 April 2005







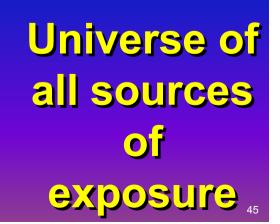




Summing up the basic concepts

EXCLUSION

(de mínímís non curat lex)



exposures deemed to be amenable to regulatory control

Universe of all sources of exposure 46

exposures deemed to be amenable to regulatory control

Excluded exposures

Universe of all sources of exposure

Basic Safety Standards:

"Any exposure whose magnitude or likelihood is essentially unamenable to control through the requirements of the Standards is deemed to be excluded from the Standards"

"unamenable" can be taken to mean that regulatory control is

- impossible
- unfeasible
- generically unwarranted

EXCLUSION

The BSS provide examples of:

- impossibility of control
 - (potassium-40 in the body)
- unfeasibility of control
 - (cosmic ray exposure at the earth's surface)
- generically unwarranted control
 - (unmodified concentrations of radionuclides in most raw materials)

SCOPE AND EXCLUSION

Scope can be specified by stating what is excluded

However, it can also be specified by defining

what is included

- this may be more pragmatic!

EXEMPTION

(de mínímís non curat prætor)

exposures deemed to be amenable to regulatory control

Exemption

Regulated Sources

Exempted

Sources

Excluded exposures

Universe of all sources of exposure 52

09/03/2009

EXEMPTION

- General principles agreed over 20 years ago:
 - the radiation risks to individuals should be sufficiently low as to be of no regulatory concern.
 - the collective should be sufficiently low as unwarranting regulatory control under the prevailing circumstances.
 - inherently safe



IAEA SAFETY GUIDES

Principles for the Exemption of Radiation Sources and Practices from Regulatory Control

INTERNATIONAL ATOMIC ENERGY AGENCY, VIENNA

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EXEMPTION

General numerical criteria agreed over 20 years ago:
 Trivial individual risk

 (10μSv/y →1mSv/y)

 Optimized radiation protection

 (around 1 man Sv)

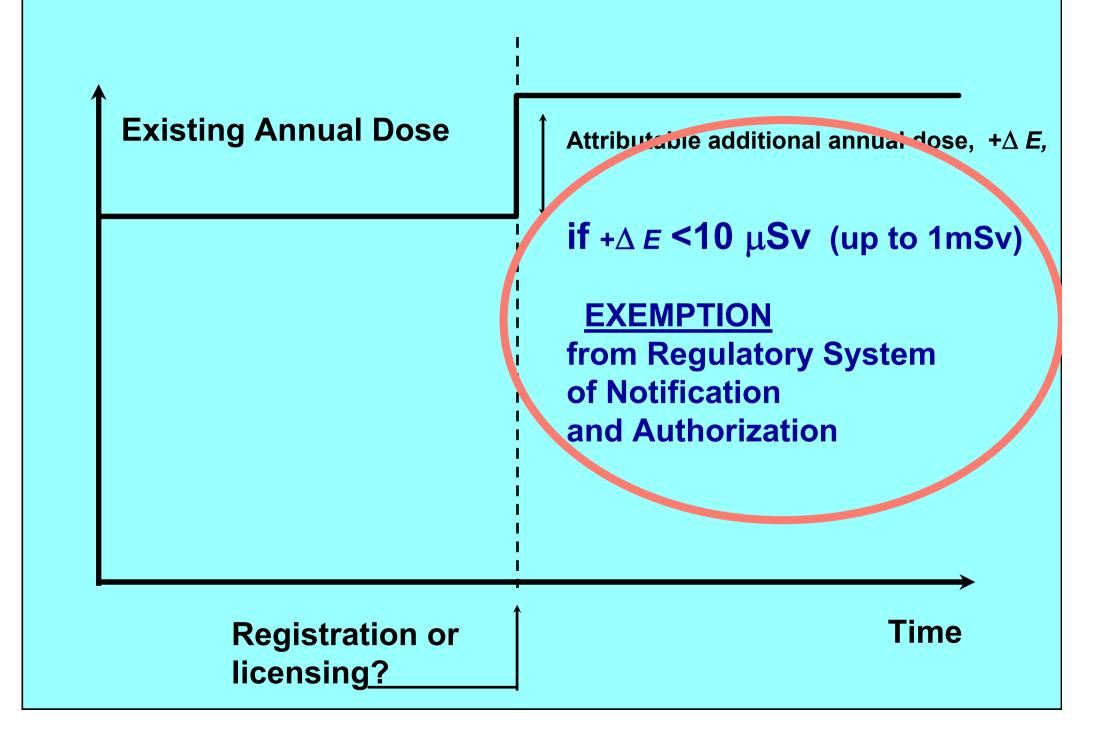
ADINTLY SPONSORED BY

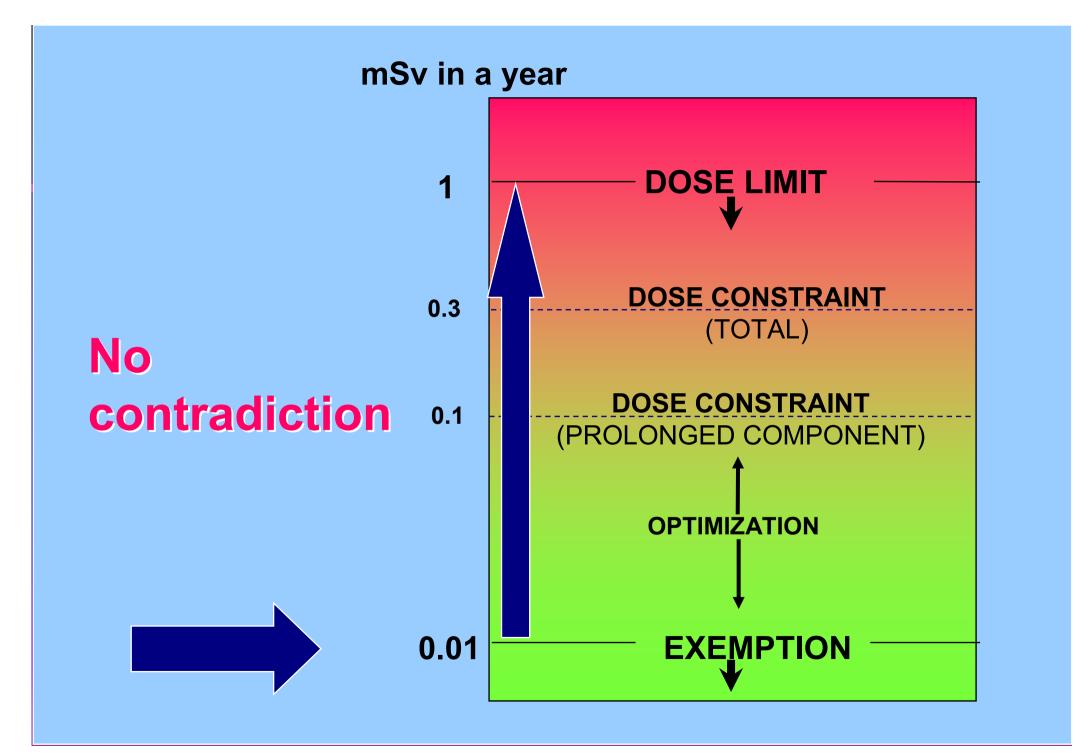
Individual risk basis

• Work of the European Communities:

Normal situations: around above 10 µSv/y

Pessimistic situations: distribution up to 1 mSv/y





International exemption

levels were established in

Schedule 1 of BSS

Safety series

International Basic Safety Standards for Protection against Ionizing Radiation and for the Safety of Radiation Sources

SAFETY

JOINTLY SPONSORED BY FAO. IAEA. ILO. OECD/NEA. PAHO, WHO





INTERNATIONAL ATOMIC ENERGY AGENCY, VIENNA, 1996

International exemption levels

- Activity
- Activity concentration (except bulk amounts)
- Energy

(radiation generators with e < 5keV)

Dose rate
 (radiation generators delivering dose rate < 1µSv h⁻¹ at 0.1m)

CLEARANCE:

exit from the system (the least 'clear' concept)

CLEARANCE

• Confusing, untranslatable terminology!

Word with unclear meaning in English

Translated into French as 'liberation'

Translated into Spanish as 'dispensation'

(and sometimes as declassification)

Clearance

"Sources, including substances, materials and objects, within notified or authorized practices may be released from further requirements of the Standards subject to complying with clearance levels approved by the **Regulatory Authority.**"

Safety Series

STANDARDS

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AF

(4)

Basic Safety Standards for Protection against Ionizing Radiation and for the Safety of Radiation Sources

INTERNATIONAL ATOMIC ENERGY AGENCY, VIENNA, 1996

CONFUSION

Is 'clearance' equivalent to

• an exemption from within?

or

• an authorized release?

'LIBERATION'

authorized level for discharges of radioactive

materials into the environment?

• If the concept is used as 'liberation', i.e., authorization of release into the environment, why is needed at all? Why the established \bigcirc concept of authorized release level is not used?



Proceedings of an International Conference, Córdoba, Spain, 13 – 17 March 2000



09/03/2009

'DISPENSATION' (Dispensa) or Declassification (desclasificación)

Original intention of the BSS drafters:

complement to exemption; i.e.:

exemption: permission not to enter into the system

<u>clearance</u>: permission to exit the system) or

EXEMPTION FROM WITHIN THE SYSTEM

Radioactivity Level	Region of Prohibited Releases	Regulatory Control
Authorized Release Limit ➡		Prohibition of Release
	Region of Authorized Releases	Authorization of Release with Increasing Conditions
'Clearance' level —→		Authorization for exiting the system
BELOW 'CLEARANCE' LEVELS	Region of 'cleared' materials	NO REGULATORY CONTROL

CLEARANCE LEVELS

The BSS states that clearance levels should not be

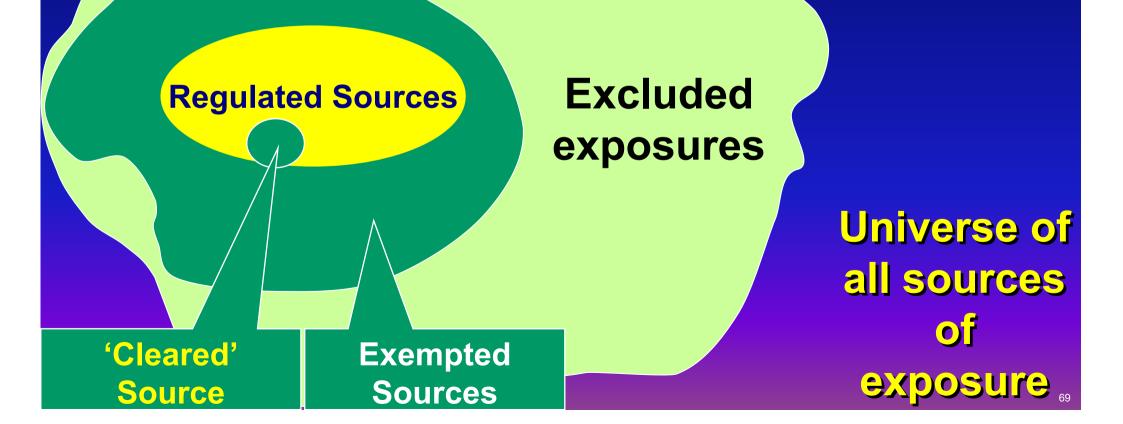
higher than the exemption levels

Bulk quantities are involved, whereas the exemption

levels were based on small quantities

exposures deemed to be amenable (in principle) to regulatory control

CLEARANCE



Summary of the Conceptual Situation

EXCLUSION

EXCLUSION

CLEARED

EXEMPTION REGULATED

a Regulatory Domain/itv

Amenable to Radiation Protection





71

Styles

"Exempters" - retain as far as possible legal instruments for control of activities involving exposure to radiation. Minimal use of exclusion, preference for exemption (by regulatory decision).

"Excluders" - only regulate activities that need to be brought into the scope of regulation. Preference for exclusion when appropriate, but also make full use of exemption.

[The terms "exempters" and "excluders" were invented for this presentation only and have no other currency]

NO INTERVENTION

Annals of the ICRP

ISSN 0146-6453

Volume 29 Nos. 1-2 1999

PUBLICATION 82

Protection of the Public in Situations of Prolonged Radiation Exposure

The Application of the Commission's System of Radiological Protection to Controllable Radiation Exposure Due to Natural Sources and Long-lived Radioactive Residues



INTERVENTION ALMOST ALWAYS JUSTIFIABLE

CRITERIA FOR INTERVENING

100

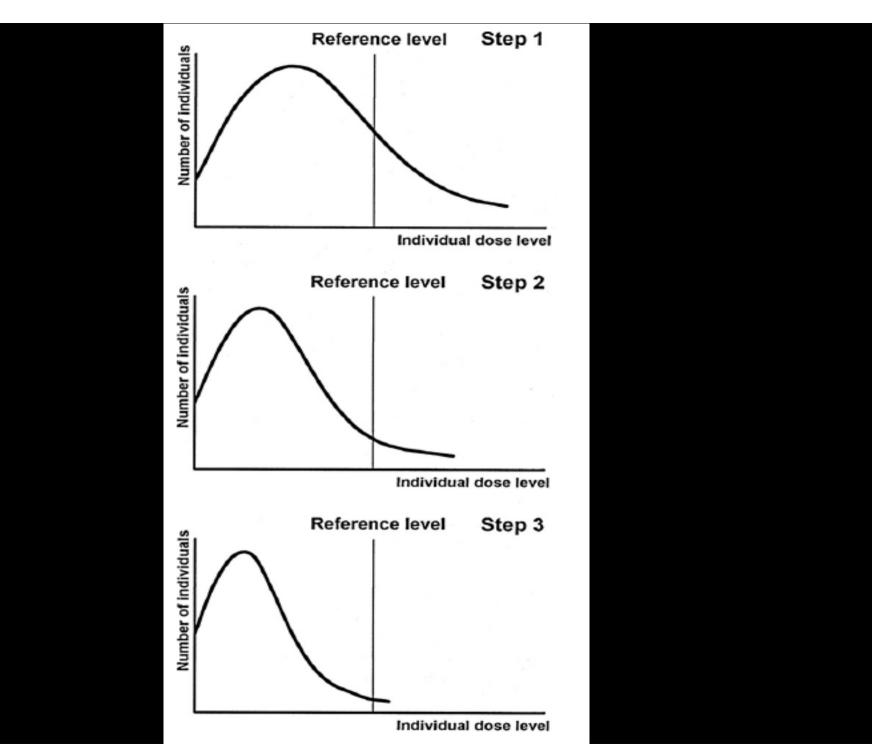
1

mSv in a year

(Extant Annual Dose)

INTERVENTION MAY BE JUSTIFIABLE

INTERVENTION IS NOT LIKELY TO BE JUSTIFIABLE



• Exclusion

unamenable control
of radiation exposure

Exemption

unwarranted control of new and inherently safe radiation sources

• Clearance

_removable controlfrom materials already under control

• No Intervention

unjustified control

of existing situations

Implications for scrap metal:

Control and management of inadvertent radioactive material

BASIC SUBDIVISION

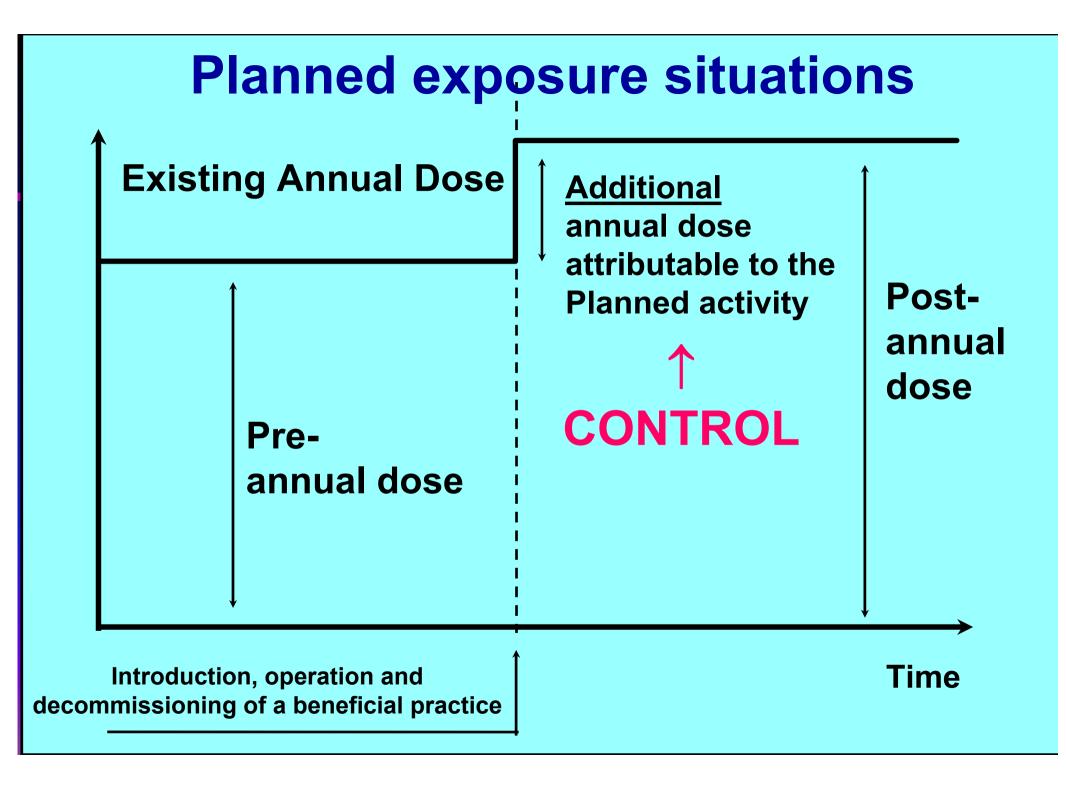
Planned exposure situations

prospective activities expecting to add radiation

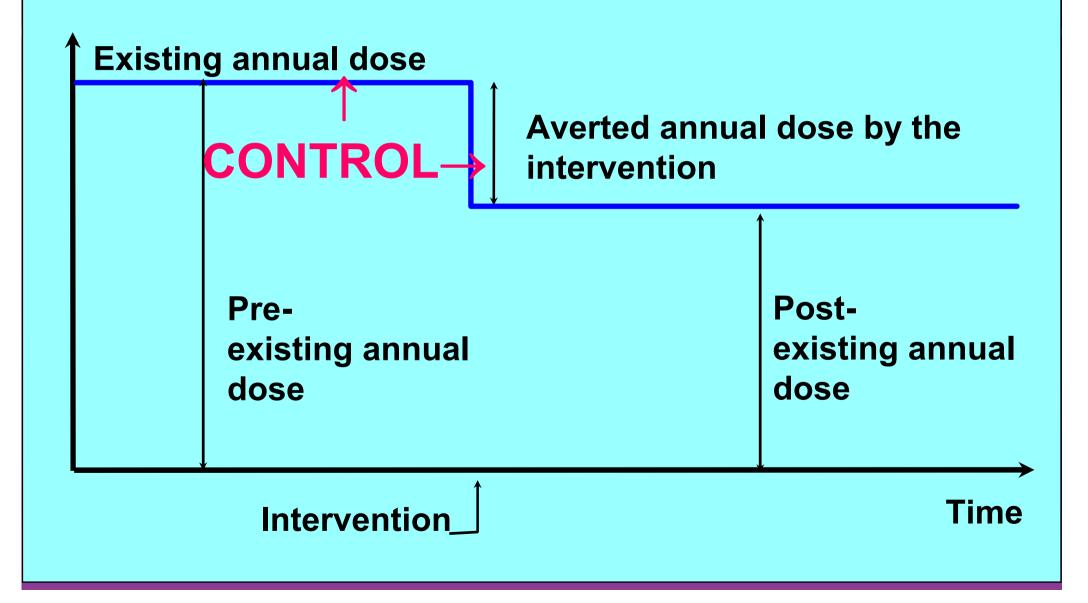
exposure

Extant and emergency exposure situations

protective actions to reduce exposure



Extant and emergency exposure situations



THEORY

• If the scrap is attributable to a planned exposure situations,

control through dose constraints

In other cases,

subject to reference levels

DILEMMA

Due to the globalization of markets,

acceptable levels:

cannot be established on a case-by-case basis

need to be <u>standardized</u>

The political

interest...

...*and*...

the time for



THE GENERAL CONFERENCE OF

THE INTERNATIONAL ATOMIC ENERGY AGENCY



Board of Governors General Conference

GOV/2004/54-GC(48)/8

Date: 30 July 2004 General Distribution Original: English

For official use only

Item 3(c) of the Board's provisional agenda (GOV/2004/51) Item 13 of the Conference's provisional agenda (GC(48)/1)

Measures to Strengthen International Co-operation in Nuclear, Radiation and Transport Safety and Waste Management

Radiological Criteria for Radionuclides in Commodities

Report by the Director General

Suggested Approach

Define the scope of regulatory control through a

simple set of

radionuclide-specific levels

of

activity concentration

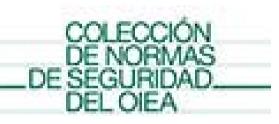


Application of the Concepts of Exclusion, Exemption and Clearance

SAFETY GUIDE

No. RS-G-1.7





Aplicación de los conceptos de exclusión, exención y dispensa





09/03/2009

SAFETY GUIDE RS-G-1.7

The values of activity concentration provided in this Safety Guide do not apply to foodstuffs, drinking water, animal feed and any material intended for this use.

TABLE 2. VALUES OF ACTIVITY CONCENTRATION FOR RADIONUCLIDES OF ARTIFICIAL ORIGIN IN BULK (see para. 4.4)

Radic- nuclide	Activity concen- tration (Bq/g)		Radio- nuclide	Activity concen- tration (Bq/g)	 Radio- nuclide	Activity concen- tration (Bq/g)	
H-3	100		Mn-56	10	Se-75	1	
Be-7	10		Fe-52	10	 Br-82	1	
C-14	1		Fe-55	1000	Rb-86	100	
F-18	10	•	Fe-59	1	Sr-85	1	
Na-22	0.1		Co-55	10	 Sr-85m	100	•
Nn-24	1	•	Co-56	0.1	Sr-87m	100	•
Si-31	1000	•	Co-57	1	Sr-89	1000	
P-32	1000		Co-58	1	Sr-90	1	
P-33	1000		Co-58m	10000	 Sr-91	10	•
S-35	100		Co-60	0.1	Sr-92	10	•
C1-36	1		Co-60m	1000	 Y-90	1000	
CI-38	10	•	Co-61	100	Y-91	100	
K-42	100		Co-62m	10	Y-91m	100	•
K-43	10	•	Ni-59	100	Y-92	100	•
Ca-45	100		Ni-63	100	Y-93	100	•
Ca-47	10		Ni-65	10	Zr-93	10	•
Sc-46	0.1		Cu-64	100	Zr-95	1	
Sc 47	100		Zn-65	0.1	Zr-97	10	•
Sc.48	1		Zn-69	1000	Nb-93m	10	
V-48	1		Zn-69m	10	 Nb-94	0.1	
Cr-51	100		Ga-72	10	Nb-95	1	
Mn-51	10	•	Ge-71	10008	Nb-97	10	•
Mn-52	1		As-73	1000	Nb-98	10	•
Mn-52m	10		As-74	10	 Mo-90	10	•
Mn-53	100		As-76	10	Mo-93	10	
Mn-54	0.1		As-77	1000	Mo.99	10	

TABLE 2. VALUES OF ACTIVITY CONCENTRATION FOR RADIONUCLIDES OF ARTIFICIAL ORIGIN IN BULK (see para. 4.4) (cont.)

()								
Radio- nuclide	Activity concen- tration (Bq/g)		Radio- nuclide	Activity concen- tration (Bq/g)		Radio- nuclide	Activity concen- tration (Bq/g)	
Mo-101	10	0	Sn-125	10		Cs-129	10	
Te-96	1		Sb-122	10		Cs-131	1000	
Tc-96m	1000	•	Sb-124	1		C⊱132	10	
Tc-97	10		Sb-125	0.1		Cs-134	0.1	
Te-97m	100		Te-123m	1		Cs-134m	1800	•
Tc-99	1		${\rm Te-125}m$	1000		Cs-135	100	
Te-99m	100		Te-127	1000		Cs-136	1	
Ru-97	10		Te-127m	10		Cs-137	0.1	
Ru-103	1		Te-129	100	+	Cs-138	10	•
Ru-105	10		Te-129m	10		Ba-131	10	
Ru-106	0.1		Te-131	100		Ba-140	1	
Rh-103m	10000		Te-131 m	10		La-140	1	
Rh-105	100		Te-132	1		Ce-139	1	
Pd-103	1000		Te-133	10		Cc-141	100	
Pd-109	100		Te-133m	10	+	Ce-143	10	
A_{B} -105	1		Te-134	10		Ce-144	10	
Ag-110m	0.1		1-123	100		Pr-142	100	٠
Ag-111	100		1-125	100		Pr-143	1000	
Cd-109	1		1-126	10		Nd-147	100	
Cd-115	10		1-129	0.01		Nd-149	100	٠
Cd-115m	100		1-130	10		Pm-147	1000	
In-111	10		1-131	10		Pm-149	1000	
In-113m	100		I-132	10		Sm-151	1000	
In-114m	10		I-133	10		Sm-153	100	
In-115m	100		I-134	10	+	Eu-152	0.1	
Su-113	1		I-135	10		Eq.152m	100	٠

TABLE 2. VALUES OF ACTIVITY CONCENTRATION FOR RADIONUCLIDES OF ARTIFICIAL ORIGIN IN BULK (see para. 4.4) (cont.)

ActivityActivityActivityRadio- nuclideconcen- ration (Bq/g)Radio- concen- nuclideRadio- concen- nuclideRadio- tration nuclideEu-1540.1Ir-1921Pa-23010Eu-1551Ir-194100*Pa-23310Gd-15310Pt-19110U-23010	n- n
Eu-1551Ir-194100*Pa-23310Gd-15310Pt-19110U-23010	
Gd-153 10 Pt-191 10 U-230 10	
Gd-159 100 * Pt-193m 1000 U-231 100	
Tb-160 1 Pt-197 1000 * U-232 0.1	
Dy-165 1000 * Pt-197m 100 * U-233 1	
Dy-166 100 Au-198 10 U-236 10	
Ho-166 100 Au-199 100 U-237 100	
Er-169 1000 Hg-197 100 U-239 100	
Er-171 100 * Hg-197m 100 U-240 100	
Tm-170 100 Hg-203 10 Np-237 1	
Tm-171 1000 Tl-200 10 Np-239 100	
Yb-175 100 T1-201 100 Np-240 10	
Lu-177 100 T1-202 10 Pu-234 100	
Hf-181 1 T1-204 1 Pu-235 100	
Ta-182 0.1 Pb-203 10 Pu-236 1	
W-181 10 Bi-206 1 Pu-237 100	
W-185 1000 Bi-207 0.1 Pu-238 0.1	
W-187 10 Po-203 10 * Pu-239 0.1	
Re-186 1000 Po-205 10 * Pu-240 0.1	
Re-188 100 * Po-207 10 * Pu-241 10	
Os-185 1 At-211 1000 Pu-242 0.1	
Os-191 100 Ra-225 10 Pu-243 1000	
Os-191m 1000 * Ra-227 100 Pu-244 0.1	
Os-193 100 Th-226 1000 Am-241 0.1	
Ir-190 1 Th-229 0.1 Am-242 1000	٠

TABLE 2. VALUES OF ACTIVITY CONCENTRATION FOR RADIONUCLIDES OF ARTIFICIAL ORIGIN IN BULK (see para. 4.4) (cont.)

Radio- nuclide	Activity concen- tration (Bq/g)		Activity concen- tration (Bq/g)	Radio- nuclide	Activity concen- tration (Bq/g)	
Am-242m	0.1	Cm-248	0.1	Cf-253	100	
Am-243	01	Bk-249	100	CE-254	1	
Cm-242	10	CI-246	1000	Es-253	100	
Cm-243	1	Cf-248	1	Es-254	0.1	
Cm-244	1	Cf-249	0.1	Es-254m	10	
Cm-245	0.1	CI-250	1	Fm-254	10000	•
Cm-246	0.1	CI-251	0.1	Fm-255	100	•
Cm-247	0.1	CI-252	1			

* Half-life of less than 1 day.

Radionuclides			
I-129	0.01		
No. 22, Sc-46; Mn-54; Co-56; Co-60; Zn-65; Nb-94; Ru-106; Ag-110m; Sb-125; Cs-134; Cs-137 Eu-152; Eu-154; Ta-182; Bi-207; Th-229; U-232; Pu-238; Pu-239; Pu-240; Pu-242; Pu- 244; Am-241; Am-242m; Am-243; Cm-245; Cm-246; Cm-247; Cm-248; Cf-249; Cf-251; Es-254	0.1		
C-14; Na-24; Cl-36; Sc-48; V-48; Mn-52; Fe-59; Co-57; Co-58; Se-75; Br-82; Sr-85; Sr-90; Zr- 95; Nb-95; Tc-96; Tc-99; Ru-103; Ag-105; Cd-109; Sn-113; Sb-124; Te-123m; Te-132; Cs-136; Ba-140;La-140;Ce-139; Eu-155; Tb-160; Hf-181; Os-185; Ir-190 Ir-192 Tl-204; Bi-206; Th- 232 ¹ , U-233; U-235 ² ; U-238 ³ Np-237; Pu-236; Cm-243; Cm-244; Cf-248; Cf-250; Cf-252; Cf-254	1		
Be-7; F-18; Cl-38; K-40; K-43; Ca-47; Mn-51; Mn-52m; Mn-56; Fe-52; Co-55; Co-62m; Ni-65; Zn-69m; Ga-72; As-74; As-76; Sr-91; Sr-92; Zr-93; Zr-97; Nb-93m; Nb-97; Nb-98; Mo-90; Mo-93; Mo-99; Mo-101; Tc-97; Ru-97; Ru-105; Cd-115; In-111; In-114m; Sn-125; Sb-122; Te-127m; Te-129m; Te-131m; Te-133; Te-133m; Te-134; I-126; I-130; I-131; I-132; I-133; I-134; I-135; Cs-129; Cs-132; Cs-138; Ba-131; Ce-143; Ce-144; Gd-153; W-181; W-187; Pt-191; Au-198; Hg-203; Tl-200; Tl-202; Pb-203; Po-203; Po-205; Po-207; Ra-225; Pa-230; Pa-233; U-230; U-236; Np-240; Pu-241; Cm-242; Es-254m	10		
H-3; S-35; K-42; Ca-45; Sc-47; Cr-51; Mn-53; Co-61; Ni-59; Ni-63; Cu-64; Rb-86; Sr-85m; Sr- 87m; Y-91; Y-91m; Y-92; Y-93; Tc-97m; Tc-99m; Rh-105; Pd-109; Ag-111; Cd-115m; In- 113m; In-115m; Te-129; Te-131; I-123; I-125; Cs-135; Ce-141; Pr-142; Nd-147; Nd-149; Sm- 153; Eu-152m; Gd-159; Dy-166; Ho-166; Er-171; Tm-170; Yb-175; Lu-177; Re-188; Os-191; Os-193; Ir-194; Pt-197m; Au-199; Hg-197; Hg-197m; Tl-201; Ra-227; U-231; U-237; U-239; U- 240; Np-239; Pu-234; Pu-235; Pu-237; Bk-249; Cf-253; Es-253; Fm-255	100		
Si-31; P-32; P-33; Fe-55; Co-60m; Zn-69; As-73; As-77; Sr-89; Y-90; Tc-96m; Pd-103; Te-125m; Te-127; Cs-131; Cs-134m; Pr-143; Pm-147; Pm-149; Sm-151; Dy-165; Er-169; Tm-171; W-185; Re-186; Os-191m; Pt-193m; Pt-197; At-211; Th-226; Pu-243; Am-242; Cf-246	1000		
Co-58m; Ge-71; Rh-103m; Fm-254	10 000		

Mixtures of Radionuclides

• The following formula should be used:

$$\sum_{i=1}^{n} \frac{C_i}{(\text{activity concentration})}_i \le 1$$

Where:

- C_i is the concentration (Bq/g) of the *i*th radionuclide of artificial origin in the material,
- (activity concentration)_i is the value of activity concentration for the radionuclide *i* in the material and
- n is the number of radionuclides present.

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> Editor J. VALENTIN

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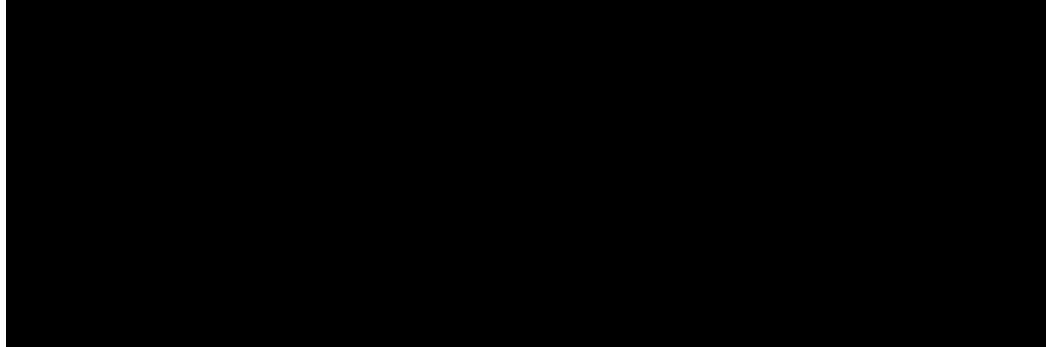
Conundrum

How to solve the problem of satisfying the clearance criterion while considering the uncertainties of the measurement and the nuclide spectrum?



Radiation Safety Research Center

Mr. Takatoshi Hattori



Paragraph (95)<u>The Commission recognizes that there may be</u> uncertainty (or variation) in the radionuclide composition of a material. In such a case, there are some concerns that the public could be exposed to a dose above the dose criterion for exemption without further consideration (10 microSv/year), although this has quite a low probability of occurring. However, in the derivation of exemption levels in the BSS (IAEA, 1996) and in the safety guide on the application of the concepts of exclusion, exemption, and clearance (IAEA, 2004b), which were agreed internationally, two dose criteria were used; 0.01 mSv/year for realistic scenarios and 1 mSv/year for lowprobability scenarios. This indicates that the exemption levels agreed under the aegis of intergovernmental organisations allow the possibility of doses greater than 10 microSv/year in the case of low-probability situations. In this regard, the Commission considers that, in cases of uncertainty (or variation) in the radionuclide composition of a material, there is not usually a need to make clearance levels stricter. However, if the uncertainties in nuclide composition are very large, or if the presence of alpha- and beta-emitting nuclides cannot be adequately inferred through gamma measurements, the regulatory body may establish specific criteria for clearance, or may demand assessments involving radionuclide analysis in addition to, or in place of, gamma measurements.

Epilogue

How is the situation today?

Much better than in 2000: an intergovernmental

agreement on clearance levels exists!!

• However:

inconsistencies and unnecessary complications

diaspora of national criteria

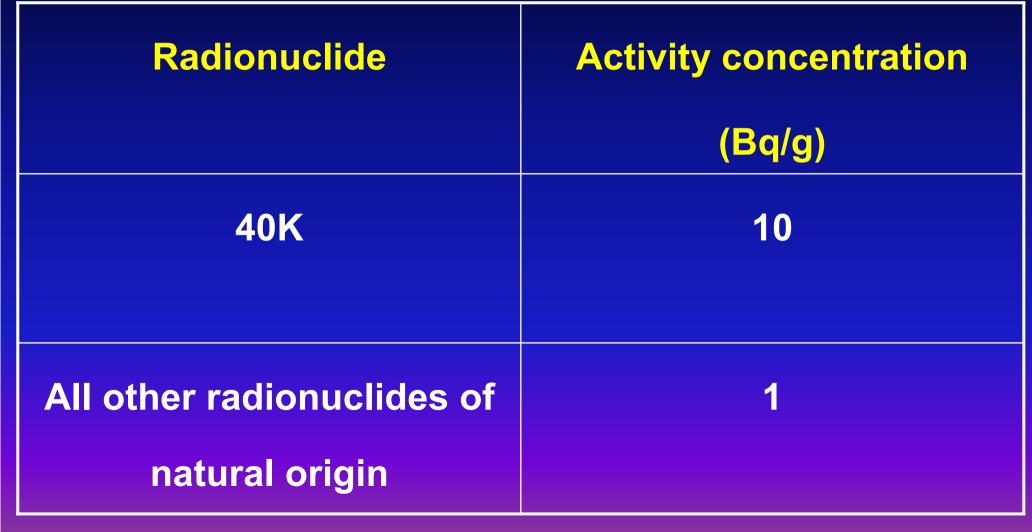
Artificial Nuclides

(Numerical Examples)

Radio-	Dangerous	Exemption levels	Clearance levels <1 ton	Clearance levels (bulk)
nuclide (examples)	Activity	Activity	Activity concentration	Activity concentration
	ТВq	Bq	Bq/g	Bq/g
Co-60	0.03	100 000	10	0.1
Cs-137	0.1	10 000	10	0.1
lr-192	0.08	10 000	10	1
Am-241	0.06	10 000	1	0.1

Natural Nuclides

Clearance levels



Are we ready for a simplifying and

rationalizing revolution?



Let's start negotiating an international protocol, including legal obligations and simple straightforward technical criteria, such

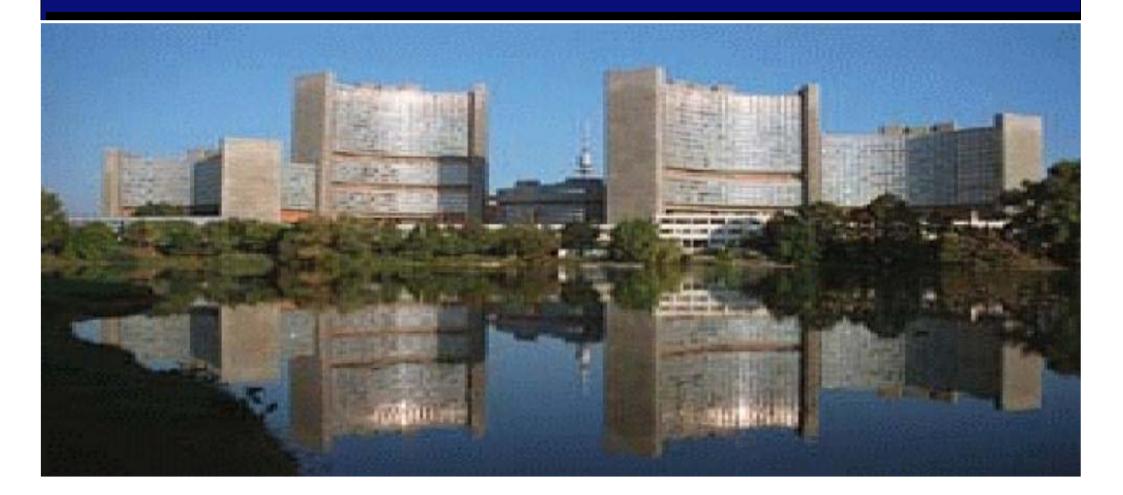
as:

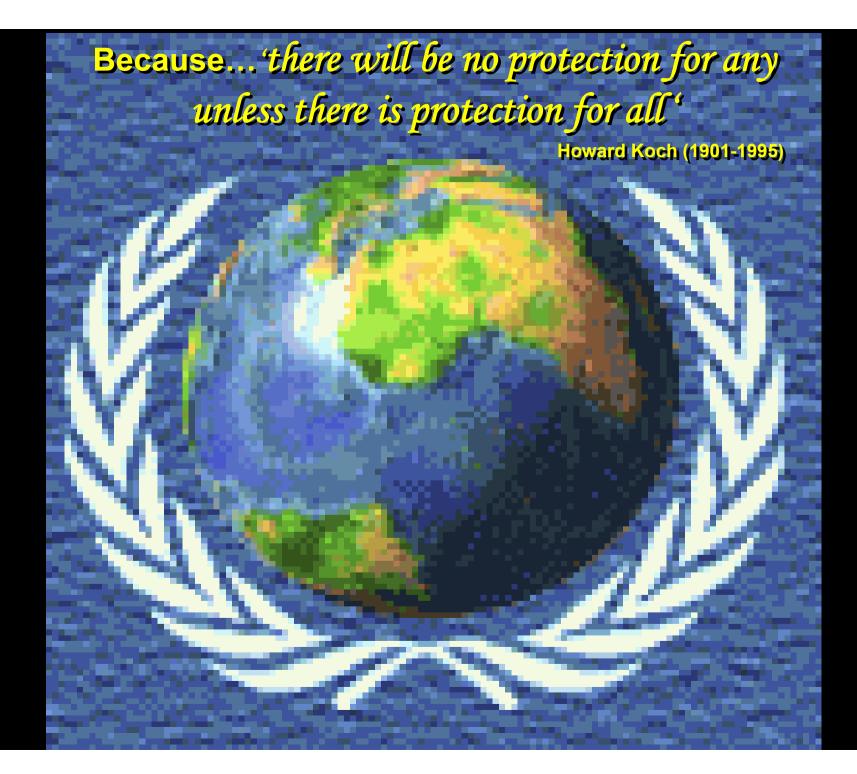
 Any substance/material with activity density lower than 0.1 - 1Bq/g is considered *de minimis not curat lex* and therefore tailored to *exclusion* from radiation protection legislation.

 Any commodity with activity density around
 1 - 10 Bq/g is considered *de minimis not curat prætor* and therefore subject to *a priori* exemption or *a posteriori* clearance by regulators.

Any commodity with activity density above
 10 - 100 Bq/g should be registered before the regulator

...and this could be an international binding undertaking under the aegis of the IAEA!

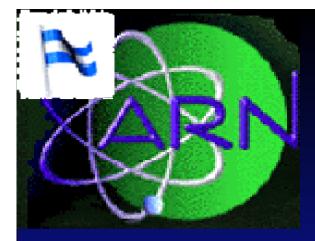




My last rumination

Exclusion and exemption Bring to me as a temptation, Reminiscences of a lawyer named Rex, Who was sadly deficient in sex, But when confronted with exposure, He declared with composure, 'de minimis not curat Rex'







Thank you for your tolerance!

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