Contributions of U.S. National Laboratories to International Nuclear Safety



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- Global expansion of nuclear energy underway
- Many new nuclear nations have no regulatory infrastructure
- Modern reactor designs marketed to wide range of customers
 - Wide range of regulatory infrastructure
 - Large need for technical support, tools and training
 - Safety culture development
- Energy security depends on uniform application of best practices and standards
 - "an accident anywhere, is an accident everywhere"
- DOE Laboratories can provide needed technical support



Operational Data Collection and Analysis

Reactor Data

Power Reactor					Event Number: 45624	
Facility: INDIAN POINT Region: Toster: NY Unit [21] [1] RX Type [2] W+4-LP NRC Hotlind By: JOHN DIGNAM HO O'S Office: JOE O'HARA					Notification Date: 01/11/2010 Notification Time: 18:33 [ET] Event Date: 01/11/2010 Event Time: 15:59 [EST] Last Update Date: 01/11/2010	
Emerge 10 CFR 50.72(b 50.72(b	ncy Class: NON Section:)(2)(Iv)(B) - RPS)(3)(Iv)(A) - VALI	EMERGENCY ACTUATION - D SPECIF SYS	CRITICAL SACTUATION	F	erson (Organization): NTHONY DIMITRIAD	IS (R1DO)
Unit	SCRAM Code	RX CRIT	Initial PWR	Initial RX Mode	Current PWR	Current RX Mode
2	AIR	Y	100	Power Operation	0	Hot Standby
AUTOM "At 155! into the maintair plant ele "Indian	ATIC TRIP DUE on January 11, generator trip is ning steam gener ectrical lineups ar Point Unit 3 was	TO MAIN GEN 2010, Indian F ongoing. All sy rator water lew re normal. No not affected an	VERATOR ELECTR Point Unit 2 tripped in Instems responded a els. Decay heat rem primary or seconda nd remains at 100%	ICAL TRIP from 100% power due to a is expected. The auxiliary f roval is via the steam gene ry side relief valves lifted. T power."	main generator electr eedwater system resg rators to the main cor he reactor plant is in	ical trip. The investigation conded as expected and is idenser. Offsite power and mode 3 and stable.
The lice	nsee notified the	NRC Residen	t Inspector and the	N.Y. State Public Service C	ommission. The licer	see intends to notify the

- Operating Experience (LERs, AITs, ENs, EPIX from INPO, etc.)
- Fire DB (Industry Cooperative Project, New Metric, Updated IEs)
- HERA Human Event Repository & Analysis
 Database
- Computational Support
 - PRA Data Collection and Basic Event Parameter Updates
 - CCF Data Collection and CCF Parameter Estimates
 - System Performance Trending of Operational Performance
- Industry Trending Annual Report to Congress on Health of Nuclear Industry
- NMED Nuclear Materials Events Database
 - Loss/Theft/Abandonment; Medical Misadministrations; Releases/Contaminations; Fuel Cycle Facility Events, etc.



Figure 7. General Event Information.



Argonne Support to NRC and International Community: Melt Coolability and Concrete Interaction (MCCI) Program

- Work organized by the OECD.
- Participating countries: Belgium, Czech Republic, Finland, France, Germany, Hungary, Japan, Norway, South Korea, Spain, Sweden, Switzerland, and the United States of America.
 - All experiments are conducted at Argonne
 - NRC functions as the project Operating Agent.
- Current program focus is on ex-vessel debris coolability
 - Viewed internationally as an important technical challenge impacting accident management strategy for LWR plants

Post-test Debris from Core-Concrete Interaction Tests



SNL Foreign Regulatory Support

Technical support contractor to Argentina Regulatory Nuclear (ARN) Authority

- Licensing of Atucha-II
 - One-of-a-kind heavy water reactor
 - Complex severe accident issues
- Expert review of Vendor Licensing Documents
 - Application of IAEA regulatory guides (IAEA-TECDOC-1229: "Regulatory review of probabilistic safety assessment (PSA) Level 2"
 - PSA Level I-II interface
 - PSA Level II
 - PSA Level III





International Cooperation on Reactor Safety Research **Phebus Testing Program and MELCOR Validation**





- Fission heating
- Pre-irradiated fuel
- Melt progression
- Hydrogen generation
- Fission product release
- Deposition in RCS
- Containment behavior
- **All Phebus experiments**
 - Documentation lagging



NRC-OECD Spent Fuel Pool Experiments

- NRC research into pool-draining accidents
- Electrically heated full scale prototypic fuel assemblies
- Measuring thermal-hydraulic performance under air natural draft conditions leading to Zr-fire
- Models implemented into MELCOR
- Test results are impacting practices at nuclear power plants in the US leading to safer operations under accident scenarios







Fuel Damaged by Zr-fire



Fuel Assembly Lower Tie Plate



MELCOR and MACCS Safety Tools Used Worldwide





Expertise and capabilities in DOE national laboratories' are available to regulators and industries in other countries



- Computational simulation
- Experimentation and model validation
- Methodology development
- Regulatory infrastructure



- Great Value in Collaborations of International Regulatory Bodies and Associated TSOs
- Needs of Each Regulatory Body Must Determine TSO Support Arrangements
- International Commitment to Research Collaboration Must Continue

* From my talk presented at April 2007 IAEA Conference on this subject